Mr. John J. Barton  
Acting Director of TMI-2  
GPU Nuclear Corporation  
P.O. Box 480  
Middletown, PA 17057

Dear Mr. Barton:

SUBJECT: TMI-2 Fuel Accountability

Subsequent to the November 4, 1981 meeting between NRC and GPU on fuel accountability, the NRC has developed more definitive guidelines for accountability during defueling of the Unit 2 core. In reviewing available information regarding the expected forms, quantities, and condition of special nuclear material (SNM), we have determined that fuel accountability at the site can be limited to item control. In this regard, a count of the assemblies or groups of assemblies that can be removed as a unit should be made. For that material that cannot be removed as a unit, a net weight of each shipment should be obtained.

Since the damaged reactor core will be transferred to DOE custody for study and evaluation, measurements for SNM content can be made at the receiver's site, where facilities are available.

For the physical protection of the SNM during transportation to the DOE facility, the requirements of 10 CFR 73.37, "Requirements for Physical Protection of Irradiated Reactor Fuel In-Transit," must be followed. Specific deviation from certain requirements may be given on a case-by-case basis, if justified in writing to the NRC.

Sincerely,

Bernard J. Snyder, Program Director  
Three Mile Island Program Office  
Office of Nuclear Reactor Regulation

cc: H. Feinroth, DOE  
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See attached list

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