

February 19, 1985

Docket No. 50-320

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Dea: Mr. Standerfer:

Subject: Three Mile Island Nuclear Station, Unit 2
Operating License No. DPR-73
Docket No. 50-320
Corrected Pages for December 19, 1984 Order

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Docket No. 50-320
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On December 19, 1984, the NRC issued an Amendment of Order modifying certain pages of the TMI-2 Proposed Technical Specifications (PTS) and the Recovery Operations Plan (ROP). In Enclosures 3 and 4 of the December Order, the staff provided modified pages of the PTS and the ROP. It has been brought to our attention that some of those pages were incorrect. Therefore, we have provided the correct pages herein. The enclosed pages should replace those previously transmitted on December 19, 1984.

Sincerely,

/s/ R. A. Weller for

Bernard J. Snyder, Program Director
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Enclosures:
As stated

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LIMITING CONDITIONS FOR OPERATION

3.4 REACTOR COOLANT SYSTEM

REACTOR COOLANT LOOPS

3.4.1 The Reactor Coolant System shall be operated in accordance with procedures approved pursuant to Specification 6.8.2.

APPLICABILITY: RECOVERY MODE.

ACTION:

None except as provided in Specification 3.0.3.

REACTOR VESSEL WATER LEVEL MONITORING

3.4.2 As a minimum two independent reactor vessel level monitoring instruments shall be OPERABLE.

APPLICABILITY: RECOVERY MODE WITH THE RV HEAD REMOVED

ACTION

- a. With only one reactor vessel level monitoring instrument OPERABLE, terminate all activities involving changes in the reactor coolant system water volume, restore the system to OPERABLE status within 72 hours.
- b. With no reactor vessel level monitoring instrument OPERABLE, terminate all activities involving changes in the reactor coolant system water volume. Restore the system to OPERABLE status within 24 hours or, in lieu of any other report required by 10 CFR 50.73, prepare and submit a Special Report to the Commission pursuant to Specification 6.9.2 within the next 30 days outlining the action taken, the cause of the inoperability and the plans and schedule for restoring the system to OPERABLE status.

SAFETY VALVES

3.4.3 Deleted.

3.4.9 PRESSURE/TEMPERATURE LIMITS

REACTOR COOLANT SYSTEM

3.4.9.1 The Reactor Coolant System shall be maintained between a T_{avg} of less than 200°F and greater than 50°F.*

3.4.9.2 The Reactor Coolant System shall remain open to the reactor building atmosphere unless repressurization is approved in a safety evaluation submitted to the NRC. This safety evaluation and associated procedures approved pursuant to Specification 6.8.2 shall specify the maximum pressure limits and over-pressure protection that is required.

*All makeup pumps shall be made inoperable by racking out their electrical power supply circuit breakers.

LIMITING CONDITIONS FOR OPERATION

3.7.6 FLOOD PROTECTION

3.7.6.1 Flood protection shall be provided for all Safety Related systems, components and structures when the water level of the Susquehanna River exceeds 301 feet Mean Sea Level USGS datum, at the river water intake structure of Three Mile Island Nuclear Station, Unit 1.

APPLICABILITY: At all times.

ACTION:

- a. With the water level at the Unit 1 Intake Structure approaching 301 feet Mean Sea Level USGS datum:
 1. Initiate patrol and inspection of the dikes surrounding the site for signs of deterioration such as undermining or excessive seepage.
 2. Inform the Site Operations Director (SOD) and as directed by the SOD:
 - a) Prepare all flood panels and door seals for installation,
 - b) Check all building floor drains and pumps to ensure proper operation,
 - c) Commence daily soundings of the Intake Screen House Floor,
 - d) Check all water tight doors to ensure proper operation,
 - e) Fill all outdoor storage tanks to inhibit floatation, and
 - f) Arrange for alternate supplies of diesel fuel oil and ensure fuel storage tanks are filled.
 - g) Check that all containers are sealed and secure in the Southeast Storage Facility.
- b. With the water level at the Unit 1 Intake Structure exceeding 301 feet and approaching 302 feet Mean Sea Level USGS datum:
 1. Ensure all door seals and flood panels are installed and all water tight doors are closed within 2 hours,
 2. Inform the Director Site Operations.

SURVEILLANCE REQUIREMENTS

4.1 WATER INJECTION COOLING AND REACTIVITY CONTROL SYSTEMS

4.1.1 BORATION CONTROL

BORATED COOLING WATER INJECTION

4.1.1.1 Two systems capable of injecting borated cooling water into the Reactor Coolant System shall be demonstrated OPERABLE:

- A. At least once per 12 hours (when system is in operation) by verifying that the Standby Reactor Coolant System Pressure Control System:
 - 1. Charging Water Storage Tank water volume is filled with a minimum of 2300 gallons.
 - 2. Deleted
 - 3. Deleted
- B. At least once per 7 days by verifying that the Charging Water Storage Tank contains borated water with:
 - 1. A boron concentration of between 3500 and 6000 ppm.
 - 2. Deleted.
- C. At least once per 31 days by verifying that at least one Standby Pressure Control System Charging Pump develops a minimum flow rate of 30 gpm.
- D. By demonstrating that the Mini Decay Heat Removal System (MDHRS) is OPERABLE by performing inservice tests of each MDHRS pump and each MDHRS valve in the flow path in accordance with SECTION XI of the ASME Boiler and Pressure Vessel Code and applicable Addenda as required by 10 CFR 50, Section 50.55a(g), except where specific written relief has been granted by the Commission pursuant to 10 CFR 50, Section 50.55a(g)(6)(i).
- E. At least once per 31 days by verifying that on recirculation flow, the decay heat removal pump required by Technical Specification 3.1.1.1 develops a discharge pressure of greater than or equal to 151 psig and that each pump operates for at least 15 minutes.

TABLE 4.3-3 (Con't)RADIATION MONITORING INSTRUMENTATION SURVEILLANCE REQUIREMENTS

NOTES:

- 1) During operation of the monitored system.
- 2) If monitor becomes inoperable, repair or replace by equivalent equipment within 12 hours. If not completed within 12 hours terminate operation of the monitored system and restore the inoperable monitor(s) to operable status.
- 3) Restore the inoperable monitor(s) to operable status within 72 hours.
- 4) If ion exchange effluent monitor is inoperable, sample on 4 hour frequency for gross beta. If inoperable longer than 24 hours then terminate operation of ion exchange system and restore the inoperable monitor to operable status.
- 5) With radioactive waste in the fuel handling building.
- 6) With the required instrumentation inoperable, suspend all operations involving movement of radioactive wastes in the fuel handling building, restore the inoperable equipment to OPERABLE status within 48 hours.
- 7) With the AMS-3 inoperable, close at least one of the Reactor Building Equipment Doors and restore the inoperable equipment to operable status prior to the reopening of both Equipment Doors.
- 8) With the Reactor Vessel Flange Area Gamma Monitor inoperable suspend all operations involving CORE ALTERATIONS and restore instrument to operable status.

LIMITING CONDITIONS FOR OPERATION

3.5 COMMUNICATIONS

3.5.1 Control Room

Direct communication shall be maintained between the Control Room or the Command Center and personnel in the Reactor Building. As stated in Table 6.2-1, the additional SOL or SOL limited to fuel handling notwithstanding location, will have direct communications with personnel in the Reactor Building.

APPLICABILITY: During CORE ALTERATIONS

ACTION:

When direct communication between the Control Room or the Command Center and personnel in the Reactor Building as stated in Table 6.2-1 cannot be maintained, suspend all operations involving CORE ALTERATIONS and restore communications to OPERABLE status.

SURVEILLANCE REQUIREMENTS

4.5 COMMUNICATIONS

4.5.1 Verify that communications channels are OPERABLE between the Control Room or the Command Center and personnel in the Reactor Building once each day prior to the initiation of any activities involving CORE ALTERATIONS. Also verify that the additional SOL or SOL limited to fuel handling has OPERABLE communication channels once each day prior to the initiation of any activities involving core alterations. 1