

Docket No. 50-320

November 8, 1993

Dr. Robert L. Long
Director, Corporate Services/
Director TMI-2
GPU Nuclear Corporation
Post Office Box 480
Middletown, Pennsylvania 17057

Dear Dr. Long:

SUBJECT: ISSUANCE OF AMENDMENT NO. 46 FOR POSSESSION ONLY LICENSE NO. DPR-73
FOR THREE MILE ISLAND NUCLEAR STATION UNIT 2 (TAC NO. M87058)

The Commission has issued the enclosed Amendment No. 46 to Possession Only License No. DPR-73 for the Three Mile Island Nuclear Station Unit 2 (TMI-2). This amendment is in response to your application of June 29, 1993 (Technical Specification Change Request No. 70).

This amendment modifies Section 3.9.12, "Fuel Handling Building/Auxiliary Building Air Cleanup Systems", and its associated basis, of the Appendix A technical Specifications by removing a reference to a section in Appendix B that was removed from Appendix B by Amendment No. 43 issued on May 26, 1993.

A copy of the related Safety Evaluation supporting this amendment is enclosed. Notice of Issuance will be included in the Commission biweekly Federal Register notice.

Sincerely,

ORIGINAL SIGNED BY
Michael T. Masnik, Senior Project Manager
Non-Power Reactors and Decommissioning
Project Directorate
Division of Operating Reactor Support
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 46 to License No. DPR-73
2. Safety Evaluation

cc w/enclosures:

See next page

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UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D.C. 20548-0001

November 8, 1993

Docket No. 50-320

Dr. Robert L. Long
Director, Corporate Services/
Director TMI-2
GPU Nuclear Corporation
Post Office Box 480
Middletown, Pennsylvania 17057

Dear Dr. Long:

SUBJECT: ISSUANCE OF AMENDMENT NO. 46 FOR POSSESSION ONLY LICENSE NO. DPR-73
FOR THREE MILE ISLAND NUCLEAR STATION UNIT 2 (TAC NO. MB705B)

The Commission has issued the enclosed Amendment No. 46 to Possession Only License No. DPR-73 for the Three Mile Island Nuclear Station Unit 2 (TMI-2). This amendment is in response to your application of June 29, 1993 (Technical Specification Change Request No. 70).

This amendment modifies Section 3.9.12, "Fuel Handling Building/Auxiliary Building Air Cleanup Systems", and its associated bases, of the Appendix A Technical Specifications by removing a reference to a section in Appendix B that was removed from Appendix B by Amendment No. 43 issued on May 26, 1993.

A copy of the related Safety Evaluation supporting this amendment is enclosed. Notice of Issuance will be included in the Commission biweekly Federal Register notice.

Sincerely,

A handwritten signature in black ink, appearing to read "Michael T. Masnik".

Michael T. Masnik, Senior Project Manager
Non-Power Reactors and Decommissioning
Project Directorate
Division of Operating Reactor Support
Office of Nuclear Reactor Regulation

Enclosures:

1. Amendment No. 46 to License No. DPR-73
2. Safety Evaluation

cc w/enclosures:
See next page

Dr. R. L. Long
GPU Nuclear Corporation Unit No. 2

Three Mile Island Nuclear Station
Docket No. 50-320

cc:

Regional Administrator, Region I
U.S. Nuclear Regulatory Commission
475 Allendale Road
King of Prussia, Pennsylvania 19406

Mr. Robert Rogan
GPU Nuclear Corporation
P. O. Box 450
Middletown, Pennsylvania 17057

Dr. Judith H. Johnsrud
Environmental Coalition on Nuclear
Power
433 Orlando Avenue
State College, Pennsylvania 16801

Mr. David J. McGoff
Office of LWR Safety and Technology
NE-23
U.S. Department of Energy
Washington, D.C. 20545

Ernest L. Blake, Jr., Esq.
Shaw, Pittman, Potts, and Trowbridge
2300 N Street, N.W.
Washington, D.C. 20037

Mr. Wythe Keever
The Patriot
812 Market Street
Harrisburg, Pennsylvania 17105

Secretary
U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

Mr. Robert B. Borsum
B & W Nuclear Technologies
Suite 525
1700 Rockville Pike
Rockville, Maryland 20852

Mr. Russell Schaeffer, Chairperson
Dauphin County Board of Commissioners
Dauphin County Courthouse
Front and Market Streets
Harrisburg, Pennsylvania 17120

Mr. Marvin I. Lewis
7801 Roosevelt Blvd. #62
Philadelphia, Pennsylvania 19152

William Dornsife, Acting Director
Bureau of Radiation Protection
Department of Environmental Resources
P. O. Box 2063
Harrisburg, Pennsylvania 17120

Mr. Jane Lee
183 Valley Road
Etters, Pennsylvania 17319

Mr. Ad Crable
Lancaster New Era
8 West King Street
Lancaster, Pennsylvania 17601

Mr. Walter W. Cohen, Consumer
Advocate
Department of Justice
Strawberry Square, 14th Floor
Harrisburg, Pennsylvania 17127

Ms. Michelo G. Evans
Senior Resident Inspector (TMI-1)
U.S. Nuclear Regulatory Commission
P. O. Box 311
Middletown, Pennsylvania 17057

U.S. Environmental Prot. Agency
Region III Office
ATTN: EIS Coordinator
841 Chestnut Street
Philadelphia, Pennsylvania 19107

Mr. Eric Epstein
2308 Brandywine Drive
Harrisburg, Pennsylvania 17110



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555-0001

GPU NUCLEAR CORPORATION

DOCKET NO. 50-320

THREE MILE ISLAND NUCLEAR STATION, UNIT NO. 2

POSSESSION ONLY LICENSE

Amendment No. 46
License No. DPR-73

- I. The U.S. Nuclear Regulatory Commission (the Commission) has found that:
 - A. The application for amendment filed by GPU Nuclear Corporation (the licensee) dated June 29, 1993, complies with the standards and requirements of the Atomic Energy Act of 1954, as amended (the Act), and the regulations of the Commission as set forth in 10 CFR Chapter I;
 - B. The facility will be maintained in conformity with the application, the provisions of the Act, and the rules and regulations of the Commission;
 - C. There is reasonable assurance (i) that the activities authorized by this amendment can be conducted without endangering the health and safety of the public, and (ii) that such activities will be conducted in compliance with the regulations of the Commission;
 - D. The issuance of this amendment will not be inimical to the common defense and security or to the health and safety of the public; and
 - E. The issuance of this amendment is in accordance with 10 CFR Part 51 of the regulations of the Commission and all applicable requirements have been satisfied.
2. Accordingly, the license is amended by changes to the Technical Specifications as indicated in the attachment to this license amendment, and paragraph 2.C(1) of Possession Only License No. DPR-73 is hereby amended to read as follows:

(1) Technical Specifications

The Technical Specifications contained in Appendices A and B, as revised through Amendment No. 46, are hereby incorporated into this license. The licensee shall maintain the facility in accordance with the Technical Specifications and all Commission Orders issued subsequent to the date of the possession only license.

3. This license amendment is effective as of the date of issuance.

FOR THE NUCLEAR REGULATORY COMMISSION



Seymour H. Weiss, Director
Non-Power Reactors and Decommissioning
Project Directorate
Division of Operating Reactor Support
Office of Nuclear Reactor Regulation

Enclosure:
Changes to the Technical
Specifications

Date of Issuance: November 8, 1993

ENCLOSURE TO LICENSE AMENDMENT NO. 46

POSSESSION ONLY LICENSE NO. DPR-73

DOCKET NO 50-320

Replace the following pages of the Appendix A Technical Specifications with the attached pages. The revised pages are identified by amendment number and contain vertical lines indicating the area of change.

Remove

3.9-2
3.9-3
B 3/4.9-1

Insert

3.9-2
3.9-3
B 3/4.9-1

LIMITING CONDITIONS FOR OPERATION

- b. With no Fuel Transfer Canal (deep end) water level instruments OPERABLE, terminate all activities involving any Canister containing core material in or over the Fuel Transfer Canal (deep end) and/or all activities involving the plenum assembly and all operations involving changes in the Fuel Transfer Canal (deep end) water inventory and restore one inoperable instrument to OPERABLE status within 24 hours.

FUEL TRANSFER CANAL (DEEP END) WATER LEVEL

3.9.4 The water level in the Fuel Transfer Canal (deep end) shall be maintained at the level specified in Section 4.9.4 of the Recovery Operations Plan.

APPLICABILITY: MODE 1

ACTION:

- a. With the Fuel Transfer Canal (deep end) water level not within the specified band, terminate all activities involving any Canister containing core material in or over the Fuel Transfer Canal (deep end) and/or all activities involving the plenum assembly and restore the water level to within specification within 24 hours.

FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEMS

3.9.12.1 The Fuel Handling Building Air Cleanup Exhaust System shall be OPERABLE with one of the four system air cleanup exhaust fans OPERABLE.

APPLICABILITY: MODES 1, 2, and 3

ACTION:

With the Fuel Handling Building Air Cleanup Exhaust System inoperable, restore the system to OPERABLE status within 4 hours, or, suspend all operations involving movement of liquid and solid radioactive wastes in the Fuel Handling Building (other than sampling evolutions required by the Technical Specifications or RECOVERY OPERATIONS PLAN), the release of which could exceed 50 percent of the OFFSITE DOSE CALCULATION MANUAL instantaneous release rate for gaseous effluents, until the system is restored to OPERABLE status.

LIMITING CONDITIONS FOR OPERATION

3.9.12.2 The Auxiliary Building Air Cleanup Exhaust System shall be OPERABLE with one of the four system air cleanup exhaust fans OPERABLE.

APPLICABILITY: MODES 1, 2, and 3

ACTION:

With the Auxiliary Building Air Cleanup Exhaust System inoperable, restore the system to OPERABLE status within 4 hours or, suspend all operations involving movement of liquid or solid radioactive wastes in the Auxiliary Building (other than sampling evolutions required by the Technical Specifications or RECOVERY OPERATIONS PLAN), the release of which could exceed 50 percent of the OFFSITE DOSE CALCULATION MANUAL instantaneous release rate for gaseous effluents, until the system is restored to OPERABLE status.

ACCIDENT GENERATED WATER

3.9.13 ACCIDENT GENERATED WATER shall be disposed of in accordance with NRC-approved procedures.

APPLICABILITY: MODES 1, 2, and 3

ACTION:

None except as provided in Specification 3.0.3.

3/4.9 RADIOACTIVE WASTE STORAGE

BASES

3/4.9.1 SPENT FUEL STORAGE POOL "A" WATER LEVEL MONITORING

Spent Fuel Storage Pool Water "A" Level Monitoring instrumentation has been provided to assure the capability to monitor water level in the Spent Fuel Storage Pool "A".

3/4.9.2 SPENT FUEL STORAGE POOL "A" WATER LEVEL

The water level in the Spent Fuel Storage Pool "A" has been established to limit the dose rate, due to the storage of Canisters, to acceptable levels.

3/4.9.3 FUEL TRANSFER CANAL (DEEP END) WATER LEVEL MONITORING

Fuel Transfer Canal Water Level Monitoring instrumentation has been provided to assure the capability to monitor water level in the deep end of the Fuel Transfer Canal.

3/4.9.4 FUEL TRANSFER CANAL (DEEP END) WATER LEVEL

The water level in the Fuel Transfer Canal (deep end) has been established to limit the dose rate, due to the storage of the plenum assemble and Canisters, to acceptable levels.

3/4.9.12 FUEL HANDLING BUILDING/AUXILIARY BUILDING AIR CLEANUP SYSTEMS

The requirement for the Fuel Handling Building/Auxiliary Building Air Cleanup System to be operating or OPERABLE ensures that radioactive material released to these buildings will be filtered through the HEPA filters prior to release to the atmosphere. In the event the systems are not restored to OPERABLE status within 4 hours, the Technical Specifications require the suspension of any liquid or solid radioactive waste handling, the release of which could exceed 50 percent of the instantaneous release rate limits for gaseous effluents specified in Part II Section 2.1.2 of the OFFSITE DOSE CALCULATION MANUAL. These restrictions correspond to solid radioactive waste with a total activity of greater than 1 curie of particulates with half-lives greater than eight (8) days and liquid radioactive wastes with an activity greater than 0.5 curies of particulates with half-lives greater than eight (8) days.

3/4.9.13 ACCIDENT GENERATED WATER

These specifications are provided to ensure compliance with the Commission's Statement of May 25, 1979, and the Commission's Statement of Policy and Notice of Intent to Prepare a Programmatic Environmental Impact Statement of November 21, 1979, which prohibit these actions pending evaluation of the environmental impacts of such actions. The PEIS issued in March 1981, deferred a decision on the ultimate disposal of processed water. Further Commission action is necessary prior to release of ACCIDENT GENERATED WATER.



UNITED STATES
NUCLEAR REGULATORY COMMISSION

WASHINGTON, D. C. 20555-0001

SAFETY EVALUATION BY THE OFFICE OF NUCLEAR REACTOR REGULATION
SUPPORTING AMENDMENT NO. 46 TO POSSESSION ONLY LICENSE NO. DPR-73

GPU NUCLEAR CORPORATION

THREE MILE ISLAND NUCLEAR STATION, UNIT 2

DOCKET NO. 50-320

1.0 INTRODUCTION

GPU Nuclear Corporation (GPUN or the licensee) requested, by letter dated March 17, 1992, (Technical Specification Change Request No. 67), with Revision 1 dated June 18, 1992, and Revision 2 dated December 23, 1992, a change to the Appendix A and Appendix B Technical Specifications for the TMI-2 plant (Facility License No. DPR-73) by relocating requirements related to radiological effluents to a new document called the Offsite Dose Calculation Manual (OOCM). On May 26, 1993, the NRC staff issued License Amendment No. 43 to GPUN approving the relocation of requirements related to radiological effluents for TMI-2.

2.0 DISCUSSION

GPUN, in a letter dated June 29, 1993, (Technical Specification Change Request No. 70) requested a change to the Appendix A Technical Specifications for TMI-2 to correct an oversight that was inadvertently omitted from Technical Specification Change Request No. 67, submitted by the licensee by letter dated March 17, 1992, as revised. Upon implementing License Amendment No. 43, issued by the NRC staff on May 26, 1993, the licensee discovered that a reference was not removed in the Appendix A Technical Specifications (ATS) to a section in the Appendix B Technical Specifications (BTS) that had been relocated to the OOCM by the license amendment. ATS Section 3.9.12 "Fuel Handling Building/Auxiliary Building Air Cleanup System", and its associated bases, Section 3/4.9.12 "Fuel Handling Building/Auxiliary Building Air Cleanup Systems", currently makes reference, in the two subsections of Section 3.9.12 and in Bases 3/4.9.12, to the "Appendix B Technical Specification Instantaneous Release Rate for Gaseous Effluents." License Amendment No. 43 relocated all reference to the instantaneous release rates for gaseous effluents in the BTS to the OOCM. The licensee proposes changing ATS Section 3.9.12, both Subsections 3.9.12.1 and 3.9.12.2, and Bases Section 3/4.9.12 to reference the instantaneous release rates for gaseous effluents currently located in the OOCM.

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3.0 EVALUATION

The NRC staff has completed its review of the licensee amendment request and has determined that this is an administrative change to correct an oversight related to a previous license amendment request. The May 26, 1993 license amendment, issued by the NRC staff, addresses the safety and environmental considerations associated with the relocation of the requirements related to radiological effluents to the ODCM. Therefore, the staff finds the proposed changes acceptable.

4.0 STATE CONSULTATION

In accordance with the Commission regulations, the State of Pennsylvania cognizant individual was notified of the proposed issuance of this amendment. The State official had no comment.

5.0 ENVIRONMENTAL CONSIDERATION

This amendment involves a change in a specific reference in the Appendix A Technical Specifications. This amendment was submitted by the licensee to correct an administrative oversight related to a previous license amendment request. The NRC staff has determined that the amendment involves no significant increase in the amounts, and no significant change in the types, of any effluents that may be released offsite, and that there is no change in individual cumulative occupational exposure or exposure to the public. The Commission has previously issued a proposed finding that the amendment involves no significant hazards consideration, and there has been no public comment on such finding (58 FR 46235) dated September 1, 1993. Accordingly, the amendment meets the eligibility criteria for categorical exclusion set forth in 10 CFR 51.22(c)(9) and 10 CFR 51.22(c)(10). Pursuant to 10 CFR 51.22(b) no environmental impact statement or environmental assessment need be prepared in connection with the issuance of this amendment.

6.0 CONCLUSION

The Commission has concluded, based on the considerations discussed above, that (1) there is reasonable assurance that the health and safety of the public will not be endangered by the proposed changes to the license, (2) such activities will be conducted in compliance with the Commission regulations, and (3) the issuance of the amendment will not be inimical to the common defense and security or the health and safety of the public.

Principal Contributor: Michael T. Masnik

Date: November 8, 1993