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NRC Form 366 (9-83)

U.S. NUCLEAR REGULATORY COMMISSION
APPROVED OMB NO. 3150-0104
EXPIRES: 8/31/85

LICENSEE EVENT REPORT (LER)

FACILITY NAME (1)
Three Mile Island, Unit 2

DOCKET NUMBER (2)
0 5 0 0 0 3 2 0 1 OF 0 1 3

TITLE (4)
Abnormal Reactor Building Sump Level Indications *due to plugged sensing lines*

EVENT DATE (5)			LER NUMBER (6)			REPORT DATE (7)			OTHER FACILITIES INVOLVED (8)								
MONTH	DAY	YEAR	YEAR	SEQUENTIAL NUMBER	REVISION NUMBER	MONTH	DAY	YEAR	FACILITY NAMES								
0	3	0	1	8	4	8	4	0	3	2	0	1	0	5	0	0	0

OPERATING MODE (9) N		THIS REPORT IS SUBMITTED PURSUANT TO THE REQUIREMENTS OF 10 CFR §. (Check one or more of the following) (11)											
POWER LEVEL (10) 0 0 0	20.402(b)	20.406(c)	50.73(a)(2)(iv)	73.71(b)									
	20.406(a)(1)(i)	50.36(e)(1)	50.73(a)(2)(v)	73.71(e)									
	20.406(a)(1)(ii)	50.36(e)(2)	50.73(a)(2)(vi)	OTHER (Specify in Abstract below and in Text, NRC Form 366A)									
	20.406(a)(1)(iii)	X 50.73(a)(2)(i)	50.73(a)(2)(vii)(A)										
	20.406(a)(1)(iv)	50.73(a)(2)(ii)	50.73(a)(2)(viii)(B)										
20.406(a)(1)(v)	50.73(a)(2)(iii)	50.73(a)(2)(ix)											

LICENSEE CONTACT FOR THIS LER (12)

NAME: Russell D. Wells, TMI-2 Licensing Engineer

TELEPHONE NUMBER: 711 794 878 4611

COMPLETE ONE LINE FOR EACH COMPONENT FAILURE DESCRIBED IN THIS REPORT (13)

CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS	CAUSE	SYSTEM	COMPONENT	MANUFACTURER	REPORTABLE TO NPROS

SUPPLEMENTAL REPORT EXPECTED (14)

EXPECTED SUBMISSION DATE (15)

MONTH: DAY: YEAR:

YES (If yes, complete EXPECTED SUBMISSION DATE) NO

ABSTRACT (Limit to 1400 spaces, i.e., approximately fifteen single-space typewritten lines) (16)

At 0830 hours on March 1, 1984, the Reactor Building (RB) Water Level Indication was declared out-of-service due to a high reading on RB Sump Level Transmitter RBS-LT-6000. Since the sensing and reference lines were calibrated and found to be accurate, the problem was believed to be due to back pressure in the sensing line as a result of blockage in the tygon tube located in the RB Sump. At 1730 hours on March 1, 1984, following several blowdowns of the sensing and reference lines, the obstruction was cleared and the level indication returned to normal. Since the 8 hour timeclock of Technical Specification 3.3.3.6 was exceeded, this event is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B). GPUNC is evaluating alternative methods of RB Sump Level Instrumentation. This LER is similar in nature to LER's 83-61, 83-17, and 82-35.

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LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

FACILITY NAME (1) Three Mile Island, Unit 2	DOCKET NUMBER (2) 0 5 0 0 0 3 2 0	LER NUMBER (6)			PAGE (3)		
		YEAR 8 4	SEQUENTIAL NUMBER 0 0 6	REVISION NUMBER 0 0			
					0 2	OF 0 3	

TEXT (If more space is required, use additional NRC Form 366A's) (17)

I. PLANT OPERATING CONDITIONS BEFORE THE EVENT

The TMI-2 facility is in a long-term cold shutdown state. The reactor decay heat is being removed via loss to ambient. Throughout this event there was no effect on the Reactor Coolant System or the core.

II. STATUS OF STRUCTURES, COMPONENTS, OR SYSTEMS THAT WERE INOPERABLE AT THE START OF THE EVENT AND THAT CONTRIBUTED TO THE EVENT

N/A

III. EVENT DESCRIPTION

At 0830 hours on March 1, 1984, the Reactor Building (RB) Water Level Indication (IEEE Code - IP) was declared out-of-service due to an abnormal (i.e., high) reading on RB Sump Level Transmitter RBS-LT-6000. This placed the unit into the Action Statement of Technical Specification 3.3.3.6 which initiated an 8 hour timeclock. In addition to the high reading on RBS-LT-6000, the flowrate through RB Sump Flow Indicator, RBS-FI-6000, was less than the required 0.6 scfm. The level transmitter and flow indicator were calibrated and found to be accurate. This indicated that the most likely cause of the high reading and low flow indication was due to back pressure in the tygon tube located in the RB Sump. A blowdown of the sensing and reference lines was then performed in accordance with the bi-weekly preventative maintenance procedure, MTX-231B. NOTE: This bi-weekly preventative maintenance procedure was initiated after the last RB Sump Level Indication failure on November 16, 1983. (Reference LER 83-61.) The blowdown, however, failed to clear the obstruction in the sensing line. Following the issuance of a Unit Work Instruction (UWI), the flow indicator was bypassed in order to utilize a higher pressure (6 psi) to blow down the lines. After several attempts, the obstruction was cleared and at 1730 hours on March 1, 1984, the RB Sump Level Indication was returned to normal. Since the 8 hour timeclock was exceeded, this event is reportable pursuant to 10 CFR 50.73(a)(2)(i)(B).

The RB Sump Level Indication was also declared out-of-service on the following times and dates: 0945 on March 10, 1984; 0744 on March 13, 1984; 0900 on March 14, 1984; and 0930 on March 18, 1984. However, in each of the above instances, the level indication was returned to normal, after blowing down the sensing and reference lines, within the 8 hour timeclock. Therefore, these events are not reportable pursuant to 10 CFR 50.73.

LICENSEE EVENT REPORT (LER) TEXT CONTINUATION

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		8 4	- 0 0 6	- 0 0	0 3	OF	0 3

TEXT (If more space is required, use additional NRC Form 366A's) (17)

The root cause of each of the above occurrences was an obstruction in the sensing line of the level transmitter. The obstruction caused a buildup of pressure in the sensing line which resulted in the high level indication.

IV. CORRECTIVE ACTIONS PLANNED

Since the bi-weekly maintenance to blow down the sensing and reference lines has not been successful in preventing obstructions from accumulating in the RB Sump Level System, GPUNC is presently evaluating alternative methods of RB Sump Level Indication. As an interim measure, a bypass valve has been installed around the RBS-FI-6000. This will enable a greater pressure to be used when blowing down the lines. The bypass valve will be closed when the level indications are taken.

V. COMPONENT FAILURE DATA

N/A

VI. AUTOMATIC OR MANUALLY INITIATED SAFETY SYSTEM RESPONSES

N/A

VII. ASSESSMENT OF THE SAFETY CONSEQUENCES AND IMPLICATIONS OF THE EVENT

There are no redundant systems available for measuring the RB Sump Level. However, during the time of this event, there were no operations in progress that would have resulted in an increase in the RB Sump Level.



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March 28, 1984

US Nuclear Regulatory Commission
Document Control Desk
Washington, DC 20555

Dear Sir:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Licensee Event Report 84-06

Attached please find Licensee Event Report 84-06 concerning an abnormal Reactor Building Sump Level Indication on March 1, 1984.

This event is considered reportable under Section 50.73(a)(2)(i)(B) of Title 10 of the Code of Federal Regulations.

Sincerely,

B. K. Kanga
B. K. Kanga
Director, TMI-2

BKK/RDW/jep

Attachments

cc: Regional Administrator - Office of I & E, Dr. T. E. Murley
Program Director - TMI Program Office, Dr. B. J. Snyder
Deputy Program Director - TMI Program Office, Mr. L. H. Barrett

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