

TRM 2 SEP 08 1983

Attachment 1
4410-83-L-0176

213
B. & W.

LICENSEE EVENT REPORT

CONTROL BLOCK: 1851129 (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | P | A | T | M | I | 2 | 2 | 0 | 0 | - | 1 | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 14 15 25 26 30 57 CAT 58

CON'T
0 1 | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 2 | 0 | 7 | 0 | 7 | 1 | 3 | 8 | 3 | 8 | 0 | 8 | 1 | 2 | 8 | 3 | 9
7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | Incore Thermocouple 0-6 began to exhibit erratic behavior; therefore, in accordance
0 3 | with Technical Specifications 3.3.3.6, Table 3.3-10, Item 10, this report is
0 4 | submitted. LER's 80-13, 80-41, 80-50, 80-53, 81-05, 81-13, 82-15, and 83-10 concern
0 5 | thermocouple failures also. This event had no adverse effects on the plant, its
0 6 | operation, or the health and safety of the public.

0 9 | X X (11) | E (12) | X (13) | I N S T R U (14) | E (15) | Z (16)
7 8 9 10 11 12 13 18 19 20

17 | LER/RO REPORT NUMBER | 8 3 (21) | 22 | 23 | 0 2 7 (24) | 26 | 27 | 0 1 (28) | 29 | I (30) | 31 | 0 (32)
32

ACTION TAKEN | FUTURE ACTION | EFFECT ON PLANT | SHUTDOWN METHOD | HOURS (22) | ATTACHMENT SUBMITTED | NPRD-4 FORM SUB. | PRIME COMP. SUPPLIER | COMPONENT MANUFACTURER
Z (18) Z (19) Z (20) Z (21) 0 0 0 0 (22) Y (23) N (24) N (25) B 1 5 5 (26)
33 34 35 36 37 40 41 42 43 44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | The reason for the failure of Thermocouple 0-6 is not known and may not be possible to
1 1 | determine given the condition of the Unit 2 core relative to incore instrumentation.
1 2 | To date, no adverse trend in the overall incore thermocouple system behavior has
1 3 | become apparent. Therefore, no further action is considered applicable.

1 5 | X (28) | 0 0 0 (29) | Recovery Mode (30) | B (31) | Operator review of data (32)
7 8 9 10 12 13 44 45 46 80

1 6 | Z (33) | Z (34) | N/A (35) | N/A (36)
7 8 9 10 11 44 45 80

1 7 | 0 0 0 (37) | Z (38) | N/A (39)
7 8 9 10 11 44 80

1 8 | 0 0 0 (40) | N/A (41)
7 8 9 10 11 44 80

1 9 | Z (42) | N/A (43)
7 8 9 10 11 44 80

2 0 | N (44) | N/A (45)
7 8 9 10 11 44 80

8308240564 830812
PDR ADOCK 05000320
5 PDR

TESS

NAME OF PREPARER Steven D. Chaplin

PHONE: (717) 948-8461

LER 83-027/01L-0
EVENT DATE - July 13, 1983

I. EXPLANATION OF OCCURRENCE

Incore Thermocouple 0-6 began to exhibit erratic behavior; therefore, in accordance with Technical Specification 3.3.3.6, Table 3.3-10, Item 10, this report is being submitted.

To date, nine (9) LER's, including this one, concern thermocouple failures. The others are LER 80-13, 80-41, 80-50, 80-53, 81-05, 81-13, 82-15, and 83-10.

There are now fourteen (14) of the fifty-two (52) incore thermocouples reported as being out-of-service (D-14, E-11, G-5, H-9, H-13, K-12, L-6, L-11, L-13, M-9, N-8, N-9, O-6 and O-12). However, three (3) of these thermocouples (including D-14, M-9, and N-9) presently appear to be functioning properly and are being used to help monitor incore condition as long as they are functioning correctly.

II. CAUSES OF THE OCCURRENCE

The precise reason for the failure/erratic behavior of Incore Thermocouple 0-6 is not known and may not be possible to determine given the condition of the Unit 2 core relative to incore instrumentation.

III. CIRCUMSTANCES SURROUNDING THE OCCURRENCE

At the time of the occurrence, the Unit 2 facility was in a long-term cold shutdown state. The reactor decay heat was being removed via loss to ambient. Throughout the event there was no effect on the Reactor Coolant System or the core.

IV. CORRECTIVE ACTIONS TAKEN OR TO BE TAKEN

Incore Thermocouple 0-6 was checked to ensure that the problem is not in any component that is accessible for repairs.

To date, no adverse trend in the overall incore thermocouple failure rate has become apparent. Therefore, no further action is considered applicable.

V. COMPONENT FAILURE DATA

The failed thermocouple was a Type K (Chromium/Alumel) thermocouple, Model No. DAZA-76-7R-1B-1T-1C, supplied by Babcock and Wilcox, manufactured by Bel Fab, Inc.

SEP 08 1983

B. & W.



GPU Nuclear Corporation
Post Office Box 480
Route 441 South
Middletown, Pennsylvania 17057
717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

August 12, 1983
4410-83-L-0176

Office of Inspection and Enforcement
Attn: Mr. Thomas E. Murley
Regional Administrator
Region I
US Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Sir:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Licensee Event Report 83-027/01L-0

Attached please find Licensee Event Report 83-027/01L-0 concerning the failure of Incore Thermocouple 0-6 on July 13, 1983.

This event is a violation of Section 3.3.3.6, Table 3.3-10, Item 10 and is reportable under Section 6.9.1.8 of the Interim Recovery Technical Specifications.

Sincerely,

B. K. Kanga
B. K. Kanga
Director, TMI-2

BKK/SDC/df

Attachments

CC: Mr. L. H. Barrett, Deputy Program Director - TMI Program Office
Dr. B. J. Snyder, Program Director - TMI Program Office

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