

Reactor building swap level indicator fails
AUG 12 1983

198

LICENSEE EVENT REPORT

Attachment 1
4410-83-L-0131

B&W

CONTROL BLOCK: 184105 1

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | P | A | T | M | I | 2 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 14 15 25 26 30 57 CAT 58

CON'T
01 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 2 | 0 | 7 | 0 | 5 | 2 | 7 | 8 | 3 | 8 | 0 | 6 | 2 | 7 | 8 | 3 | 9
7 8 80 81 68 69 74 75 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES 10

02 | At 1200 hours on May 27, 1983, it was recognized that the Reactor Building water level
03 | instrumentation indication had not changed all week despite the addition of approximately
04 | 24,000 gallons (~3.5") of water. The instrument was declared out-of-service, the
05 | level instrument examined, repaired, and returned to service at 1635 hours on May 27,
06 | 1983. This event is considered reportable pursuant to Tech Spec 6.9.1.9(b) due to
07 | entry into and compliance with the Action Statement of Tech Spec 3.3.3.6. This event
08 | had no effect on the health and safety of the public.

09 | SYSTEM CODE | I | E | 11 | CAUSE CODE | E | 12 | CAUSE SUBCODE | B | 13 | COMPONENT CODE | V | A | L | V | E | X | 14 | COMP. SUBCODE | A | 15 | VALVE SUBCODE | D | 16 |
17 | LER/RO REPORT NUMBER | EVENT YEAR | 8 | 3 | 22 | SEQUENTIAL REPORT NO. | 0 | 1 | 7 | 24 | OCCURRENCE CODE | 0 | 3 | 28 | REPORT TYPE | L | 30 | REVISION NO. | 0 | 32 |
ACTION TAKEN | X | 18 | FUTURE ACTION | C | 19 | EFFECT ON PLANT | Z | 20 | SHUTDOWN METHOD | Z | 21 | HOURS | 0 | 0 | 0 | 0 | 22 | ATTACHMENT SUBMITTED | Y | 23 | NPRD-4 FORM SUB. | N | 24 | PRIME COMP. SUPPLIER | L | 25 | COMPONENT MANUFACTURER | P | 0 | 7 | 0 | 26 |

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS 27

10 | This event was the result of improper seating of the level instrument equalizing valve
11 | After flushing and cycling the valve apparently resealed properly and the instrument
12 | indication returned to normal. The system was then returned to operation. The
13 | equalizing valve was subsequently replaced.

15 | FACILITY STATUS | X | 28 | % POWER | 0 | 0 | 0 | 29 | OTHER STATUS | Recovery Mode | 30 | METHOD OF DISCOVERY | A | 31 | DISCOVERY DESCRIPTION | Operator observation | 32 |
16 | ACTIVITY RELEASED OF RELEASE | Z | 33 | CONTENT | Z | 34 | AMOUNT OF ACTIVITY | N/A | 35 | LOCATION OF RELEASE | N/A | 36 |
17 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | N/A | 39 |
18 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | N/A | 41 |
19 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | N/A | 43 |
20 | PUBLICITY ISSUED | N | 44 | DESCRIPTION | 8307110441 830627 PDR ADDCK 05000320 S PDR | 45 | NRC USE ONLY

NAME OF PREPARER: Steven D. Chaplin PHONE: (717) 948-8461

LER 83-017/03L-0
EVENT DATE - May 27, 1983

I. EXPLANATION OF OCCURRENCE

At 1200 hours on May 27, 1983, it was recognized that the Reactor Building basement water level instrument indication had not changed all week despite the addition of approximately 24,000 gallons (~3.5") to the Reactor Building basement via decon operations. The level instrument was declared out-of-service and a work request was submitted to investigate the cause. In checking the instrument, it was determined that the instrument's equalizing valve was leaking by. The valve was opened, purged, and reclosed. After flushing and reclosing the indicated level increased to 12" (the calculated level). The Reactor Building sump level indicator was returned to service at 1635 hours on May 27, 1983, within the 8 hour Tech Spec time clock.

This event is considered reportable pursuant to Tech Spec 6.9.1.9(b) due to entry into and compliance with the Action Statement of Tech Spec 3.3.3.6.

II. CAUSE OF THE OCCURRENCE

This event was the result of improper seating of the level instrument equalizing valve.

III. CIRCUMSTANCES SURROUNDING THE OCCURRENCE

At the time of the occurrence, the Unit 2 facility was in a long-term cold shutdown state. The reactor decay heat was being removed via loss to ambient. Throughout the event there was no effect on the Reactor Coolant System or the core.

IV. CORRECTIVE ACTIONS TAKEN OR TO BE TAKEN

Immediate

Visual checks of the Reactor Building basement water level, the Reactor Building side of the bubbler tube and areas outside of the Reactor Building basement were performed to ensure that the Reactor Building was not leaking.

Troubleshooting of the bubbler system began, including purging of the bubbler and instrument lines and valving. This resulted in the discovery of the leaking equalizing valve.

Long-Term

The equalizing valve was subsequently replaced.

V. COMPONENT FAILURE DATA

Manufacturer: Parker Hannifin -- Parker Fluid Connectors
Model: 3/8" V6LJ Series Stainless Steel Ball Valve

B&W AUG 12 1983



GPU Nuclear Corporation
Post Office Box 480
Route 441 South
Middletown, Pennsylvania 17057
717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

June 27, 1983
4410-83-L-0131

Office of Inspection and Enforcement
Attn: Mr. Thomas E. Murley
Regional Administrator
Region I
US Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

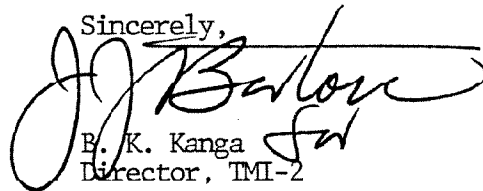
Dear Sir:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Licensee Event Report 83-017/03L-0

Attached please find Licensee Event Report 83-017/03L-0 concerning the inoperability of the Reactor Building water level instrument on May 27, 1983.

This event concerns Section 3.3.3.6 and is considered reportable under Section 6.9.1.9(b) of the Interim Recovery Technical Specifications.

Sincerely,



B. K. Kanga
Director, TMI-2

BKK/SDC/jep

Attachments

CC: Mr. L. H. Barrett, Deputy Program Director - TMI Program Office
Dr. B. J. Snyder, Program Director - TMI Program Office

IE 22
11