Update on fire watch not performed on hourly basis

NRC FORM 366 (7-77) UPDATE REPORT - PREVIOUS REPORT DATE 3/14/83

U. S. NUCLEAR REGULATORY COMMISSION

LICENSEE EVENT REPORT

OCT 10 1983

Attachment 2 B&W Three Mile Island 2 227

CONTROL BLOCK: 1 | 1 8 5 9 1 4 1 4 | 6 (1)

(PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

0 1 | P A T M I 2 (2) | 0 0 - 0 0 0 0 0 - 0 0 (3) | 4 1 1 1 1 (4) | (5)

7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T

0 1 | REPORT SOURCE L (6) | 0 5 0 0 0 3 2 0 (7) | 0 2 1 1 8 3 (8) | 0 3 0 5 8 3 (9)

7 8 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

0 2 | Between February 1 and 7, 1983, while replacing selected fire barrier seals in the

0 3 | Unit 2 River Water Pump House, fire watch tours were performed on a less than hourly

0 4 | basis during the backshifts (1500-2300 and 2300-0700 hours) and during weekend shifts.

0 5 | This is contrary to the requirements of section 3.7.11 and is, therefore, reportable

0 6 | pursuant to Section 6.9.1.8(b) of the Recovery Technical Specifications. This event

0 7 | had no effect on the health and safety of the public.

0 8 | 7 8 9 80

	SYSTEM CODE	CAUSE CODE	CAUSE SUBCODE	COMPONENT CODE	COMP. SUBCODE	VALVE SUBCODE
0 9	A B (11)	A (12)	A (13)	P E N E T R (14)	X (15)	Z (16)
	9 10	11	12	13 18	19	20

	LER/RO REPORT NUMBER (17)	EVENT YEAR		SEQUENTIAL REPORT NO.		OCCURRENCE CODE	REPORT TYPE		REVISION NO.
		8 3	—	0 0 3	/	0 1	X	—	1
		21 22	23	24 26	27	28 29	30	31	32

Rev. 0 on file

12-6-83

	ACTION TAKEN	FUTURE ACTION	EFFECT ON PLANT	SHUTDOWN METHOD	HOURS (22)	ATTACHMENT SUBMITTED	NPRD-4 FORM SUB.	PRIME COMP. SUPPLIER	COMPONENT MANUFACTURER
	H (18)	X (19)	Z (20)	Z (21)	0 0 0 0	X (23)	N (24)	Z (25)	Z 9 9 9 (26)
	33	34	35	36	37 40	41	42	43	44 47

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

1 0 | Personnel error was the proximate cause. The root cause involves the assignment of

1 1 | priorities such that proper personnel resources were not applied. The Shift Foreman

1 2 | was made aware of the problem and the fire watch was implemented on an hourly basis.

1 3 | A new Fire Watch Documentation Sheet has been developed to document the required fire

1 4 | watches.

7 8 9 80

	FACILITY STATUS	% POWER	OTHER STATUS (30)	METHOD OF DISCOVERY	DISCOVERY DESCRIPTION (32)
1 5	X (28)	0 0 0 (29)	Recovery Mode	C (31)	Quality Assurance Monitoring
	9	10 12	13 44	45	46 80

	ACTIVITY RELEASED	CONTENT OF RELEASE	AMOUNT OF ACTIVITY (35)	LOCATION OF RELEASE (36)
1 6	Z (33)	Z (34)	N/A	N/A
	9	10 11 44		45 80

	PERSONNEL EXPOSURES NUMBER	TYPE	DESCRIPTION (39)
1 7	0 0 0 (37)	Z (38)	N/A
	9 11	12 13	80

	PERSONNEL INJURIES NUMBER	DESCRIPTION (41)
1 8	0 0 0 (40)	N/A
	9 11	12 80

	LOSS OF OR DAMAGE TO FACILITY TYPE (42)	DESCRIPTION (43)
1 9	Z	N/A
	9 10	80

	PUBLICITY ISSUED (44)	DESCRIPTION (45)	NRC USE ONLY
2 0	N	N/A	
	9 10	68 69	80

NAME OF PREPARER Russ Wells PHONE: (717) 948-8461

GPO 917-926

LICENSEE EVENT REPORT
NARRATIVE REPORT
TMI-II
LER 83-03/01X-1
EVENT DATE - February 23, 1983

I. EXPLANATION OF OCCURRENCE

GPUNC has been replacing selected fire barrier seals since mid January, 1983. While each seal is being replaced, it is considered to be non-functional. Recovery Technical Specification 3.7.11 requires the performance of an hourly fire watch in the area of the impaired seal(s). A Quality Assurance monitoring of security gate access records and other plant records for the period of February 1 through February 7 showed that while replacing seals in the Unit 2 River Water Pump House, the fire watch was performed, on average, only two (2) or three (3) times per shift during the backshifts (1500-2300 and 2300-0700 hours) and during the weekend shifts.

This event is a violation of Technical Specification 3.7.11 due to exceeding the Action Statement requirements and is, therefore, reportable pursuant to Technical Specification 6.9.1.8(b).

II. CAUSE OF THE OCCURRENCE

Personnel error was the proximate cause of this event. The root cause involves the assignment of priorities such that proper personnel resources were not applied. The Operations personnel relied on the fire detection system to alert them to a problem in the screen house in lieu of the hourly inspection.

III. CIRCUMSTANCES SURROUNDING THE OCCURRENCE

At the time of the occurrence, the Unit 2 facility was in a long-term cold shutdown state. The reactor decay heat was being removed via loss to ambient. Throughout the event there was no effect on the Reactor Coolant System or the core.

IV. CORRECTIVE ACTIONS TAKEN OR TO BE TAKEN

Immediate

The Shift Foreman was made aware of the situation by the issuance of a Quality Deficiency Report (QDR). The hourly surveillance was implemented and the information was passed on to the remaining shifts via the Shift Foreman's Shift Relief Checklist.

Long-Term

In order to prevent occurrences of the above event, a Fire Watch Documentation Sheet has been developed in order for Control Room Operators to document who performed the hourly fire watch. This new form is to be attached to the applicable Auxiliary Operator Log Sheet when a fire watch is required.

V. COMPONENT FAILURE DATA

N/A

OCT 10 1983

B. & W.

GPU Nuclear

GPU Nuclear Corporation
Post Office Box 480
Route 441 South
Middletown, Pennsylvania 17057
717 944-7621
TELEX 84-2386
Writer"s Direct Dial Number:

August 5, 1983
4410-83-L-0163

Office of Inspection and Enforcement
Attn: Dr. Thomas E. Murley
Regional Administrator
US Nuclear Regulatory Commission
Region I
631 Park Avenue
King of Prussia, PA 19406

Dear Sir:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Updated Licensee Event Reports

The Licensee Event Reports listed in Attachment 1 have been updated and are enclosed as Attachment 2 to this letter.

If you have any questions, please contact Mr. J. J. Byrne of my staff.

Sincerely,

B. K. Kanga

B. K. Kanga
Director, TMI-2

BKK/RDW/jep

Attachments

CC: Mr. L. H. Barrett, Deputy Program Director - TMI Program Office
Dr. B. J. Snyder, Program Director - TMI Program Office

IE22
1/1

Attachment 1
4410-83-L-0163

LIST OF UPDATED LICENSEE EVENT REPORTS

LER 82-12

LER 82-16

LER 82-24

LER 82-28

LER 82-32

LER 82-33

LER 82-36

LER 83-03