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GPU Nuclear Corporation
Post Office Box 480
Route 441 South
Middletown, Pennsylvania 17057
717 944-7621
TELEX 84-2386
Writer's Direct Dial Number:

May 19, 1983
4410-83-L-0082

Office of Inspection and Enforcement
Attn: Mr. J. M. Allan
Acting Regional Administrator
Region I
US Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

Dear Sir:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Updated Licensee Event Reports

82-034

The Licensee Event Reports listed in Attachment 1 have been updated and are enclosed as Attachment 2 to this letter.

If you have any questions, please contact Mr. J. J. Byrne of my staff.

Sincerely,

A handwritten signature in cursive script, appearing to read 'B. K. Kanga'.

B. K. Kanga
Director, TMI-2

BKK/RDW/jep

Attachments

CC: Mr. L. H. Barrett, Deputy Program Director - TMI Program Office
Dr. B. J. Snyder, Program Director - TMI Program Office

8305310001 830519
PDR ADOCK 05000320
S PDR

LIST OF UPDATED LICENSEE EVENT REPORTS

<u>LER NO.</u>	<u>LER NO.</u>
80-01	81-12
80-05	81-20
80-07	81-22
80-12	81-23
80-49	81-32
80-54	81-34
80-55	81-35
80-56	81-36
80-57	81-38
81-04 *	82-34
81-08	
81-10	

* Event date on original Licensee Event Report was incorrect. This revision corrects the event date.

LICENSEE EVENT REPORT
NARRATIVE REPORT
TMI-II
LER 82-034/01X-1
EVENT DATE - October 21, 1982

I. EXPLANATION OF OCCURRENCE

This event concerns the inoperability of the "A" and "B" Once Through Steam Generator (OTSG) level indicators. On October 21, 1982, at 1500 hours, the "A" OTSG level indication failed. Investigation determined the cause to be a separated electrical connection. However, during the investigation it was determined at approximately 1800 hours on October 21, 1982, that the "B" OTSG level instrument was also inoperable due to failed electronic components. Further, at approximately 0620 hours on October 22, 1982, it was discovered that the level indicators were isolated from the OTSG's as a result of having the OTSG Recirculation System Isolation valves (GR-V-7A/B) closed. The valves were opened, the level indicators were repaired, calibrated, and returned to service at 1730 hours on October 22, 1982.

This event was a violation of Technical Specification 3.3.3.6 - Table 3.3-10 and is considered reportable under Section 6.9.1.8(b and f) of the Recovery Technical Specifications.

The component failure part of this LER is similar in nature to LER 81-33.

II. CAUSE OF THE OCCURRENCE

The causes contributing to this event are as follows:

- The "A" OTSG level indication failure has been attributed to construction activity which caused the "A" output connector (banana plug type) to be pulled from the transmitter inadvertently.
- The "B" OTSG level indication failure was attributed to a failed 12 volt D.C. power supply and its associated carrier/demodulator board.
- The valves, GR-V-7A/B, which supplied OTSG level indication were verified as being open during the August 14, 1982, performance of Surveillance Procedure 4301-M8 (Containment Integrity Verification). During the September 15, 1982, and October 13, 1982, performance of 4301-M8, the valves were recorded as closed. These valves were closed in accordance with Surveillance Procedure 4301-M8. This procedure contains notations on these valves that allow them to be open under administrative control. However, valves GR-V-7A/B were identified on the Surveillance Procedure data sheet as "Recirc System Isolation" only and did not specify their OTSG level instrument isolation function. With that and since the OTSG Recirc System is not operable, it was not recognized that these valves should have been open.

- When the "B" OTSG level indication was calibrated, it was discovered that the instrument's gain had been changed.

III. CIRCUMSTANCES SURROUNDING THE OCCURENCE

At the time of the occurrence, the Unit 2 facility was in a long-term cold shutdown state. The reactor decay heat was being removed via loss to ambient. Throughout the event there was no effect on the Reactor Coolant System or the core.

IV. CORRECTIVE ACTIONS TAKEN OR TO BE TAKEN

Immediate

The immediate corrective actions were:

- Replaced the "A" OTSG transmitter output plug into its receptacle.
- Replaced the "B" OTSG 12 volt D. C. power supply and carrier/demodulator board.
- Immediately opened valves GR-V-7A/B.
- Recalibrate the level indications in the Control Room.
- Secured OTSG level indication wiring to preclude any inadvertent bumping or removal of such wiring by personnel traffic.

Long Term

- To preclude any misunderstandings about the required positions of containment isolation valves, Surveillance Procedure 4301-M8 was revised (Rev. 10 dated 12/26/82) to reflect which valves should normally be open under Administrative Controls.
- To prevent any inadvertent changes in the indication's gain, the gain adjustment switch/control was secured.

V. COMPONENT FAILURE DATA

- 12 volt D.C. Power Supply made by Power One, Inc., and distributed by Nuclear Services Corporation, Campbell, CA (stock symbol #811-354-6500).
- Carrier/Demodulator Board distributed by Nuclear Services Corporation, Campbell, CA (stock symbol #063-389-3000).

ATTACHMENT IX

MAP-5 Sample Data Sheet
RM-A5, RM-A8, RM-A9
(circle one)

Sample Time: _____ Date: _____

Technician: _____

		Sample Results		
Sample Data		Nuclide	Activity (μci)	Concentration ($\mu\text{ci/cc}$)
Channel No. 1				
1. Sample Flow Rate (cc/min) =				
2. Elapsed Sample Time (min) =				
3. Sample Volume (cc) (1 x 2) =				
4. Contact Dose Rate (mR/hr) =		Total		
Channel No. 2				
1. Sample Flow Rate (cc/min) =				
2. Elapsed Sample Time (min) =				
3. Sample Volume (cc) (1 x 2) =				
4. Contact Dose Rate (mR/hr) =		Total		
Channel No. 3				
1. Sample Flow Rate (cc/min) =				
2. Elapsed Sample Time (min) =				
3. Sample Volume (cc) (1 x 2) =				
4. Contact Dose Rate (mR/hr) =		Total		

LICENSEE EVENT REPORT

Attachment 1

Update on SG level indicators inoperable 3 mi island 2

CONTROL BLOCK: 11829591 (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | P | A | T | M | I | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5

CON'T 01 | REPORT SOURCE | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 2 | 0 | 7 | 1 | 0 | 2 | 1 | 8 | 2 | 8 | 0 | 5 | 1 | 9 | 8 | 3 | 9

02 | On October 21, 1982, at 1500 hours the "A" Once Through Steam Generator (OTSG) level indication failed. Investigation determined the cause. In addition the investigation resulted in the determination at 1800 hours on October 21, 1982, that the "B" OTSG level indication was failed and also a determination at 0620 hours on October 22, 1982, that the level instruments were isolated from the OTSG's by the OTSG Recirculation System Isolation Valves GR-V-7A/B being closed. This event had no impact on the health and safety of the public.

09 | SYSTEM CODE | I | E | 11 | CAUSE CODE | D | 12 | CAUSE SUBCODE | Z | 13 | COMPONENT CODE | I | N | S | T | R | U | 14 | COMP. SUBCODE | P | 15 | VALVE SUBCODE | Z | 16 | LER/RO REPORT NUMBER | 17 | EVENT YEAR | 8 | 2 | SEQUENTIAL REPORT NO. | 0 | 3 | 4 | OCCURRENCE CODE | 0 | 1 | REPORT TYPE | X | REVISION NO. | 1 | ACTION TAKEN | X | 18 | FUTURE ACTION | G | 19 | EFFECT ON PLANT | Z | 20 | SHUTDOWN METHOD | Z | 21 | HOURS | 0 | 0 | 0 | 0 | ATTACHMENT SUBMITTED | Y | 23 | NPRD-4 FORM SUB. | N | 24 | PRIME COMP. SUPPLIER | L | 25 | COMPONENT MANUFACTURER | X | 9 | 9 | 9 | 26

10 | The causes have been determined to be a separated electrical connector, a failed power supply, and an inadequate procedure resulting in not identifying 2 closed valves required open respectively. Surveillance procedure performance records indicate that these valves have been closed since September 15, 1982. The valves were opened, the level indicators were repaired, calibrated, and returned to service at 1730 on October 22.

15 | FACILITY STATUS | X | 28 | % POWER | 0 | 0 | 0 | 29 | OTHER STATUS | Recovery mode | 30 | METHOD OF DISCOVERY | A | 31 | DISCOVERY DESCRIPTION | Operator observation | 32 | ACTIVITY RELEASED | Z | 33 | CONTENT OF RELEASE | Z | 34 | AMOUNT OF ACTIVITY | N/A | 35 | LOCATION OF RELEASE | N/A | 36 | PERSONNEL EXPOSURES NUMBER | 0 | 0 | 0 | 37 | TYPE | Z | 38 | DESCRIPTION | N/A | 39 | PERSONNEL INJURIES NUMBER | 0 | 0 | 0 | 40 | DESCRIPTION | N/A | 41 | LOSS OF OR DAMAGE TO FACILITY TYPE | Z | 42 | DESCRIPTION | N/A | 43 | PUBLICITY ISSUED | N | 44 | DESCRIPTION | N/A | 45

REV. 1/23/86

Rev. 0 on file

8305310114 830519 PDR ADOCK 05000320 S PDR

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