

JUN 15 1983

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Writer's Direct Dial Number:

May 19, 1983
4410-83-L-0082

80-056

Office of Inspection and Enforcement
Attn: Mr. J. M. Allan
Acting Regional Administrator
Region I
US Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, PA 19406

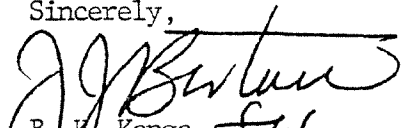
Dear Sir:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Updated Licensee Event Reports

The Licensee Event Reports listed in Attachment 1 have been updated and are enclosed as Attachment 2 to this letter.

If you have any questions, please contact Mr. J. J. Byrne of my staff.

Sincerely,


B. K. Kanga
Director, TMI-2

BKK/RDW/jep

Attachments

CC: Mr. L. H. Barrett, Deputy Program Director - TMI Program Office
Dr. B. J. Snyder, Program Director - TMI Program Office

8305310001 830519
PDR ADOCK 05000320
S PDR

IE22

LIST OF UPDATED LICENSEE EVENT REPORTS

<u>LER NO.</u>	<u>LER NO.</u>
80-01	81-12
80-05	81-20
80-07	81-22
80-12	81-23
80-49	81-32
80-54	81-34
80-55	81-35
80-56	81-36
80-57	81-38
81-04 *	82-34
81-08	
81-10	

* Event date on original Licensee Event Report was incorrect. This revision corrects the event date.

LICENSEE EVENT REPORT

Attachment 1

CONTROL BLOCK: _____ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | P | A | T | M | I | 2 | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | 5
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T
01 | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 2 | 0 | 7 | 1 | 2 | 1 | 7 | 8 | 0 | 8 | 0 | 5 | 1 | 9 | 8 | 3 | 9
7 8 REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | On December 12, 1980 during an evolution to drain the Core Floor Tanks (CFT's) it was
03 | determined that the "A" CFT Level instrumentation was inoperable. This was a viola-
04 | tion of Technical Specification 3.3.3.6 and is reported under Section 6.9.1.8(b).
05 | This event had no effect on the plant, its operation, or the health and safety of the
06 | public.
07 |
08 |
7 8 9

09 | S F | 11 | E | 12 | X | 13 | X X X X X X | 14 | X | 15 | Z | 16 | X | 17 | 8 | 0 | 18 | 0 | 5 | 6 | 19 | X | 20 | 1 | 21 | 0 | 0 | 0 | 0 | 22 | Y | 23 | N | 24 | X | 25 | X | 9 | 9 | 9 | 26
7 8 SYSTEM CODE 9 10 CAUSE CODE 11 CAUSE SUBCODE 12 COMPONENT CODE 13 COMP. SUBCODE 14 VALVE SUBCODE 15 REVISION NO. 32
LER/RO REPORT NUMBER 21 22 SEQUENTIAL REPORT NO. 24 OCCURRENCE CODE 27 REPORT TYPE 30
ACTION TAKEN 33 FUTURE ACTION 34 EFFECT ON PLANT 35 SHUTDOWN METHOD 36 HOURS 37 ATTACHMENT SUBMITTED 40 NRPD-4 FORM SUB. 42 PRIME COMP. SUPPLIER 43 COMPONENT MANUFACTURER 44
17

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | To date the cause of the instrument failure or which component has failed has not been
11 | determined. It has been determined that the failed component is within the Contain-
12 | ment Bldg. No other action is appropriate as the CFT's are no longer required by Tech.
13 | Specs. or needed for safe maintenance of the facility. Tech. Spec. 3.3.3.6 was re-
14 | vised to delete the CFT level indicators from the specifications.
7 8 9

15 | X | 28 | 0 | 0 | 0 | 29 | Recovery Mode | 30 | A | 31 | Operator observation | 32
7 8 9 FACILITY STATUS 10 % POWER 12 OTHER STATUS 30 METHOD OF DISCOVERY 45 DISCOVERY DESCRIPTION 32

16 | Z | 33 | Z | 34 | N/A | 35 | N/A | 36
7 8 9 ACTIVITY CONTENT RELEASED 33 AMOUNT OF ACTIVITY 35 LOCATION OF RELEASE 36

17 | 0 | 0 | 0 | 37 | Z | 38 | N/A | 39
7 8 9 PERSONNEL EXPOSURES NUMBER 10 TYPE 11 DESCRIPTION 39

18 | 0 | 0 | 0 | 40 | N/A | 41
7 8 9 PERSONNEL INJURIES NUMBER 10 DESCRIPTION 41

19 | Z | 42 | N/A | 43 | 8305310036 830519
7 8 9 LOSS OF OR DAMAGE TO FACILITY TYPE 10 DESCRIPTION 43 PDR ADDCK 05000320 S PDR

20 | N | 44 | N/A | 45 |
7 8 9 PUBLICITY ISSUED 10 DESCRIPTION 45 NRC USE ONLY

NAME OF PREPARER Steven D. Chaplin

PHONE: (717) 948-8461

Rev. 0 on file

LICENSEE EVENT REPORT
NARRATIVE REPORT
TMI-II
LER 80-56/01X-1
EVENT DATE - December 12, 1980

I. EXPLANATION OF OCCURRENCE

On December 12, 1980, during an evolution to drain the Core Flood Tanks (CFT's) it was determined that the "A" CFT level indication was inoperable.

After draining the "B" CFT to the "B" Reactor Coolant Bleed tank, the CFT's (A and B) were equalized. Since the "A" CFT was at a higher pressure, it discharged to the "B" CFT. The instruments for the "B" CFT indicated increased pressure and level. The pressure indicators for the "A" CFT registered a decrease, but the level indicators did not. They remained at their setting of 7.5'. Therefore, both level indicators of the "A" tank were declared inoperable. The draining evolution was then completed. This violation of Tech. Spec. 3.3.3.6 is reported in accordance with Section 6.9.1.8(b) of the Interim Recovery Technical Specifications.

II. CAUSE OF THE OCCURRENCE

The cause of the level instrumentation inoperability is not known at this time.

III. CIRCUMSTANCES SURROUNDING THE OCCURRENCE

At the time of the occurrence, the Unit II facility was in a long term cold shutdown state. The reactor decay heat was being removed via Natural Circulation to the A steam generator which is operating in a 'steaming' mode. Throughout the event there was no Loss of Natural Circulation heat removal in the RCS system.

IV. CORRECTIVE ACTIONS TAKEN OR TO BE TAKEN

IMMEDIATE

The Failure was investigated to the extent that all components in the failed instrumentation loops external to the containment were checked and found operable. Therefore, the failure occurred within containment affecting the sensor/transmitter and/or its associated wiring.

No other immediate corrective action was deemed necessary for the following reasons: 1) the CFT's are not necessary for safe operation of

the plant, 2) the CFT's are not required by the Tech. Specs., and 3) the CFT's were emptied. In addition, investigation within the containment requires work within elevated radiation areas.

LONG TERM

Tech. Spec. 3.3.3.6 was revised on April 28, 1981 to delete the CFT level indicators from the specifications as the CFT's were no longer required for safe operation of TMI-2.

V. COMPONENT FAILURE DATA

The component failure data cannot at present be determined since the failed component has not been identified.