



Metropolitan Edison Company
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Writer's Direct Dial Number

September 5, 1980
TLL 446

Office of Inspection and Enforcement
Attn: Mr. Boyce H. Grier, Director
Region I
U. S. Nuclear Regulatory Commission
631 Park Avenue
King of Prussia, Pennsylvania 19406

Dear Sir:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)
Operating License No. DPR-73
Docket No. 50-320
Licensee Event Report 80-036/01L-0

Attached please find Licensee Event Report 80-036/01L-0, concerning the failure to obtain an RCS sample analysis which was promptly reported on August 8, 1980.

This event constitutes a violation of Sections 3.1.1.2 and 3.4.9, and is considered reportable under Section 6.9.1.8(b) of the Interim Recovery Technical Specifications.

Sincerely,

G. K. Hovey
Director, TMI-2

GKH:SDC:dad

Attachments

cc: J. T. Collins
B. J. Snyder

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LICENSEE EVENT REPORT
NARRATIVE REPORT

TMI-2

LER 80-036/01L-0

EVENT DATE - August 8, 1980

I. EXPLANATION OF OCCURRENCE

On August 4, 1980, the weekly Reactor Coolant System (RCS) sample was drawn and sent to Babcock and Wilcox Research Department (B&WRD) in Lynchburg, VA. The B & W facility has been performing our weekly RCS dissolved boron and hydrogen concentration analyses since the Unit 2 facility is not equipped to perform these analyses.

On August 6, 1980, B&WRD informed Met-Ed that their facility had become contaminated to the point that they were unable to perform the analyses on our RCS sample. The possibility of having these analyses performed by another laboratory was investigated and found not to be possible within the time required. On August 8, 1980, with the sample results delinquent, the event became prompt reportable under Technical Specification 6.9.1.8(b).

II. CAUSE OF THE OCCURRENCE

This event was caused by B & W's Laboratory being too contaminated to safely perform our weekly RCS sample analysis.

III. CIRCUMSTANCES SURROUNDING THE OCCURRENCE

At the time of the occurrence, the Unit 2 facility was in a long-term cold shutdown state. The reactor decay heat was being removed via natural circulation to the "A" steam generator which is operating in a 'steaming' mode. Throughout the event, there was no Loss of Natural Circulation heat removal in the RCS System.

Although the exact RCS dissolved hydrogen and boron concentrations could not be determined by analysis during the week of 8/4/80, that does not indicate that the given concentrations were either totally unknown or unacceptable.

The only makeup to the reactor coolant is from the Standby Pressure Control System (SPC). The SPC System is maintained within the limits of 3000 to 4500 ppm boron and less than 15 cc/kg total dissolved gas was verified by on-site analysis. Since the RCS status was known for the previous week and any additions by the SPC System were within the limits, it was known that the RCS chemistry could not have been out of its limits of 3000-4500 ppm for boron. With respect to the dissolved Hydrogen, operating experience has shown that the concentration could approach, to its limit, the minimum concentration only after major evolutions. No such evolutions occurred, therefore, no deviations from the norm were expected.

B&WRD resumed the weekly analysis with the August 11, 1980, sample. The results showed boron concentration of 4100 ppm and hydrogen at 12cc/kg.

The August 4, 1980, sample was analyzed on August 15, 1980, with results of 3860 ppm for boron and 1.25 cc/kg hydrogen. This hydrogen sample is non-representative due to the 11-day wait between sampling and analysis.

IV. CORRECTIVE ACTIONS TAKEN OR TO BE TAKEN

IMMEDIATE

Upon Notification by B&WRD of their inability to perform the analysis, Met-Ed investigated the possibility of having the analyses performed by another laboratory and found that it was not possible within the required time period.

LONG TERM

Prior to this occurrence, Met-Ed initiated steps to decrease our dependence on an outside agency to perform these routine analyses. To that end, equipment necessary for on-site analyses was ordered. To date, some of that equipment has already been received. Once all the equipment is received, setup and appropriate operating and administrative controls are in place, the task of routine analyses will be performed on site.

V. COMPONENT FAILURE DATA

N/A