

JUN 15 1983



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Writer's Direct Dial Number:

May 19, 1983  
4410-83-L-0082

80-005

Office of Inspection and Enforcement  
Attn: Mr. J. M. Allan  
Acting Regional Administrator  
Region I  
US Nuclear Regulatory Commission  
631 Park Avenue  
King of Prussia, PA 19406

Dear Sir:

Three Mile Island Nuclear Station, Unit 2 (TMI-2)  
Operating License No. DPR-73  
Docket No. 50-320  
Updated Licensee Event Reports

The Licensee Event Reports listed in Attachment 1 have been updated and are enclosed as Attachment 2 to this letter.

If you have any questions, please contact Mr. J. J. Byrne of my staff.

Sincerely,

B. K. Kanga  
Director, TMI-2

BKK/RDW/jep

Attachments

CC: Mr. L. H. Barrett, Deputy Program Director - TMI Program Office  
Dr. B. J. Snyder, Program Director - TMI Program Office

8305310001 830519  
PDR ADDCK 05000320  
S PDR

IE22

LIST OF UPDATED LICENSEE EVENT REPORTS

<u>LER NO.</u>	<u>LER NO.</u>
80-01	81-12
80-05	81-20
80-07	81-22
80-12	81-23
80-49	81-32
80-54	81-34
80-55	81-35
80-56	81-36
80-57	81-38
81-04 *	82-34
81-08	
81-10	

\* Event date on original Licensee Event Report was incorrect. This revision corrects the event date.

LICENSEE EVENT REPORT

Attachment 1

CONTROL BLOCK: \_\_\_\_\_ (PLEASE PRINT OR TYPE ALL REQUIRED INFORMATION)

01 | P | A | T | M | I | 2 | 0 | 0 | - | 0 | 0 | 0 | 0 | 0 | - | 0 | 0 | 3 | 4 | 1 | 1 | 1 | 1 | 4 | \_\_\_\_\_ | 5  
7 8 9 LICENSEE CODE 14 15 LICENSE NUMBER 25 26 LICENSE TYPE 30 57 CAT 58

CON'T  
01 | L | 6 | 0 | 5 | 0 | 0 | 0 | 3 | 2 | 0 | 7 | 0 | 2 | 1 | 8 | 8 | 0 | 8 | 0 | 5 | 1 | 9 | 8 | 3 | 9  
7 8 REPORT SOURCE 60 61 DOCKET NUMBER 68 69 EVENT DATE 74 75 REPORT DATE 80

EVENT DESCRIPTION AND PROBABLE CONSEQUENCES (10)

02 | During mode 5 operation, the "B" Emergency Diesel Generator, DF-X-1B was out of ser-  
03 | vice for annual maintenance per Tech. Spec. requirements. At 1700 hours February 15,  
04 | 1980, the Interim Recovery Tech Specs were issued. The new Tech Specs required both  
05 | Emergency Diesel Generators be OPERABLE. Since the "B" unit could not and was not  
06 | returned to operable status within the action requirements, Tech. Spec. 3.8.1.1.a was  
07 | violated and reportable under Tech. Spec. 6.9.1.8.b.

08 | \_\_\_\_\_

09 | SYSTEM CODE: EE (11) CAUSE CODE: X (12) CAUSE SUBCODE: Z (13) COMPONENT CODE: Z Z Z Z Z Z (14) COMP. SUBCODE: Z (15) VALVE SUBCODE: Z (16)

17 | LER/RO REPORT NUMBER: 801 EVENT YEAR: 80 SEQUENTIAL REPORT NO.: 005 OCCURRENCE CODE: 011 REPORT TYPE: X (18) REVISION NO.: 1  
ACTION TAKEN: Z (18) FUTURE ACTION: X (19) EFFECT ON PLANT: Z (20) SHUTDOWN METHOD: Z (21) HOURS: 0000 (22) ATTACHMENT SUBMITTED: Y (23) NPRD-4 FORM SUB.: N (24) PRIME COMP. SUPPLIER: Z (25) COMPONENT MANUFACTURER: Z1999 (26)

CAUSE DESCRIPTION AND CORRECTIVE ACTIONS (27)

10 | This reportable event is the result of a change in Technical Specifications for the  
11 | Emergency Diesel Generators. The previous Tech. Specs. for mode 5 operation required  
12 | One (1) Emergency Generator be operable. The Interim Recovery Tech. Specs. require  
13 | both. Tech. Spec 3.8.1.1. was revised on August 11, 1980, in order to provide a 7-day  
14 | action time for the performance of diesel annual maintenance.

15 | FACILITY STATUS: G (28) % POWER: 000 (29) OTHER STATUS: N/A (30) METHOD OF DISCOVERY: Z (31) DISCOVERY DESCRIPTION: N/A (32)

16 | ACTIVITY CONTENT: Z (33) Z (34) AMOUNT OF ACTIVITY: N/A (35) LOCATION OF RELEASE: N/A (36)

17 | PERSONNEL EXPOSURES: 000 (37) Z (38) DESCRIPTION: N/A (39)

18 | PERSONNEL INJURIES: 000 (40) DESCRIPTION: N/A (41) 8305310007 830519 PDR AD0CK 05000320 S PDR

19 | LOSS OF OR DAMAGE TO FACILITY: Z (42) DESCRIPTION: N/A (43)

20 | PUBLICITY ISSUED: N (44) DESCRIPTION: N/A (45)

*NO Rev. 0 on file*

PO 517-126

LICENSEE EVENT REPORTNARRATIVE REPORT

TMI-II

LER 80-005/01X-1

EVENT REPORT-February 18, 1980

I. EXPLANATION OF OCCURRENCE

The B Diesel Generator has been out of service for an annual maintenance inspection which was required by the Unit II Technical Specifications. In mode 5, as defined by the previous Technical Specifications, only one diesel was required operable thus permitting the maintenance to be performed on one of the two diesels.

At 1700 hours February 15, 1980, the Unit II Recovery Technical Specifications were issued. The new specifications required both diesels to be operable and the action statement provided only 72 hours in which to have one diesel inoperable per Tech Spec 3.8.1.1 action a. Since the maintenance work took longer than 72 hours, the action was not complied with; thus this condition became reportable per Tech Spec 6.9.1.8.b.

II. CAUSE OF THE OCCURRENCE

This event was caused by the issuance of the new Technical Specification whose requirements were different from the existing Technical Specification. The B diesel inoperability was acceptable under the previous Tech Specs but not under the new specifications. Since the diesel was disassembled in such a manner that it was not possible to return it to service within the required action time the violation occurred because of the administrative transition into the new Technical Specifications.

III. CIRCUMSTANCES SURROUNDING THE OCCURRENCE

At the time of the occurrence, the Unit II facility was in a long term cold shutdown state described as Mode 5 operation by the previous Technical Specifications. The reactor decay heat was being removed via natural circulation to the A steam generator which is operating in a 'steaming' mode.

IV. CORRECTIVE ACTIONS TAKEN OR TO BE TAKEN

It is desirable to perform annual maintenance on the diesels as a matter of safety and reliability. The maintenance requires greater than 72 hours to complete. Therefore, Technical Specification 3.8.1.1 was revised on August 11, 1980, in order to provide a 7-day action time for the performance of the diesel annual maintenance.