

March 25, 1985
NRC/TMI-85-023

MEMORANDUM FOR: Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Bernard J. Snyder, Program Director
TMI Program Office

FROM: William D. Travers, Deputy Program Director
TMI Program Office

SUBJECT: NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT FOR
March 18, 1985 - March 24, 1985

1. PLANT STATUS

- The facility remains in long term cold shutdown with the Reactor Coolant System (RCS) vented to the reactor building atmosphere and the reactor vessel head removed.
- RCS cooling is by natural heat loss to the reactor building ambient atmosphere. Incore thermocouple readings range from 66°F to 89°F with an average of 80° F. Average cold leg temperature is 60°F.
- Calculated reactor decay heat is less than 14 kilowatts.
- The modified internals indexing fixture is installed on the reactor vessel flange and is flooded to elevation 327 feet 6 inches (15½ feet above the top of the core region).
- The average reactor building temperature is 58°F. The reactor building airborne activity is 4.6 E-8 uCi/cc tritium, and 1.0 E-10 uCi/cc particulates, predominantly cesium 137.

2. WASTE MANAGEMENT

- No EPICOR II processing occurred this week.
- Submerged Demineralizer System (SDS) completed processing Batch 115, which was a 52,600 gallon batch of RCS water.
- The Makeup and Purification Elution System has processed 22 batches through the "A" demineralizer and 19 batches through the "B" demineralizer.
- Total volume processed through SDS to date is 2,750,261 gallons, and the total volume processed through EPICOR II is 2,329,633 gallons.

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- Preparations are in progress for the shipment of the eighteenth SDS liner to the DOE facility at Richland, Washington. This liner was used in the processing of the waste generated by the makeup and purification elution process.

3. DOSE REDUCTION/DECONTAMINATION ACTIVITIES

- Scabbling and painting continued in the decay heat vaults on the 258' elevation of the Auxiliary Building.
- Scabbling and painting continued in the spray vaults on the 258' elevation of the Auxiliary Building.
- Reactor Building entries involving work on the 347' level have resulted in an average worker radiation dose of 34 mrem per entry man-hour. Work on the 305' level has resulted in an average dose of 100 mrem per entry man-hour.
- Average general area radiation dose rate on the 347' level of the reactor building is 36 mrem per hour. Average dose rate on the 305' level is 160 mrem per hour.

4. ENVIRONMENTAL MONITORING

- The Lancaster water sample taken at the water works intake and analyzed by the US Environmental Protection Agency consisted of a seven day composite sample taken from March 3, to March 9, 1985. A gamma scan detected no reactor related radioactivity.
- THI water samples taken by the US Environmental Protection Agency at the plant discharge to the river consisted of seven daily composite samples taken from March 2, to March 9, 1985. Gamma scans detected no reactor related radioactivity.
- The NRC outdoor air sampler at the THI Site collected a sample between March 13, 1985 and March 21, 1985. Analysis showed I-131 and Cs-137 concentrations less than the lower limits of detectability. No reactor related radioactivity was detected.
- Based on EPA's sampling results listed above, THI site liquid effluents are in accordance with regulatory limits, NRC requirements, and the City of Lancaster Agreement.

5. REACTOR BUILDING ACTIVITIES

- Wire probes were inserted into 26 of the 52 incore instrument tubes in an attempt to obtain additional data on the condition of the lower reactor vessel internals. The incore instrument tubes penetrate the bottom of the reactor vessel and extend upward into the core region. Only one of the 26 wire probes reached the reactor vessel. The remaining 25 wires were blocked at various locations along the 100

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feet of pipe between the bottom of the vessel and the pipe access at the incore seal table.

The wire probe that reached the reactor vessel was inserted 22 inches into the core region. A slightly larger diameter gamma sensor was inserted into the same instrument tube and reached the vicinity of the 5 3/8" thick lower reactor vessel head. The results of the experiments are being evaluated.

- The 25 ton capacity auxiliary hoist on the polar crane has been refurbished and is scheduled for load testing during the week of March 24, 1985. All previous lifts in the reactor building had been performed with the slow moving, 500 ton design capacity main hook.

6. AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES

- Installation of piping, pipe hangers, and electrical cables for the Defueling Water Cleanup System continued.
- Refurbishment of the "A" fuel pool continued.
- Makeup and Purification System elution continued. Water and chemicals have been added to the "A" demineralizer for the final chemical adjustment in preparation for wet layup. Water rinsing of the "B" demineralizer is in progress and will be followed by chemical adjustment.

7. NRC EVALUATIONS IN PROGRESS

- Plenum Removal Safety Evaluation: Discussions were held between NRC and licensee staff on issues relating to plenum removal. NRC review of the additional information provided is in progress.
- Fuel Canister Rack Technical Evaluation
- Defueling Water Cleanup System Technical Evaluation
- Technical Specification Change Requests number 46 and 47
- Polar Crane Auxiliary Hoist Refurbishment Plan and Load Test Safety Evaluation: Additional information has been provided by the licensee, and NRC staff review is near completion.
- Equipment Hatch Removal Safety Evaluation
- Defueling Criticality Evaluation
- Boron Dilution Analysis

8. PROJECTED SCHEDULE OF FUTURE EVENTS

- Polar Crane Auxiliary Hoist Requalification: March 28, 1985
- Plenum Removal: May 15, 1985

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9. PUBLIC MEETINGS

- The Advisory Panel for the Decontamination of Three Mile Island Unit 2 will meet on April 11, 1985, from 7:00 p.m. to 10:00 p.m. The meeting, which will be in the Lancaster Council Chambers, Public Safety Building, 201 N. Duke Street, Lancaster, Pennsylvania, will be open to the public. At the meeting, the Panel will receive presentations from the licensee on the distribution of fuel in the primary system and on plenum removal. The NRC staff will give a presentation on the staff's review of potential for inadvertent recriticality events. The NRC staff and the licensee will provide information on radiation protection issues related to the TMI-2 cleanup.

Persons desiring the opportunity to speak before the Panel are asked to contact Mr. Thomas Smithgall at 717-291-1042 or write to him at 2122 Marietta Avenue, Lancaster, Pennsylvania 17603. Persons desiring to submit topics or questions for consideration by the Panel are asked to contact, in writing, Mayor Arthur Morris, 120 North Duke Street, Lancaster, Pennsylvania 17602.

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William D. Travers
Deputy Program Director
TMI Program Office

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