

March 11, 1985
NRC/TMI-85-018

MEMORANDUM FOR: Harold R. Denton, Director
Office of Nuclear Reactor Regulation

Bernard J. Snyder, Program Director
TMI Program Office

FROM: William D. Travers, Deputy Program Director
TMI Program Office

SUBJECT: NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT FOR
March 3, 1985 - March 10, 1985

REACTOR BUILDING ACTIVITIES:

Daily reactor building entries will continue during the week of March 10, 1985. Reactor building work is focused on preparations for plenum removal in May, 1985 and commencement of defueling in late summer. Near term work inside containment includes refurbishment of the polar crane auxiliary hook, modifications to the fuel transfer system, and dismantling of one of two refueling bridges.

AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES:

Installation of pipe hangers and electrical wires for the Defueling Water Cleanup System have continued. A flush of the seal return water system reduced area radiation levels in the annulus between the Fuel Handling Building and the Reactor Building from 9000 mR/hr to nearly 300 mR/hr. The refurbishment of the "A" fuel pool has continued.

The elution process used in removing the cesium activity from the damaged makeup and purification demineralizer resins is nearing completion. Nineteen batches (~200 gallons each) have been processed through the "A" demineralizer removing about 778 curies of cesium 137. Seventeen batches have been processed through the "B" demineralizer removing about 3,379 curies of cesium 137. The current plan is to process two more batches consisting of processed water through the "A" demineralizer and to rinse the remaining elution chemicals from the resin. The "B" demineralizer will be processed with two or three more elution batches with 1.0 Normal sodium hydroxide followed by two rinse batches. Both demineralizers will then be placed in wet lay up with RCS grade water and vented to the waste gas header. The demineralizers will be stored in this condition until plans are finalized for ultimate removal and disposal of the resin. The licensee will perform surveys of the demineralizer to assess the effectiveness of the elution program.

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TME*

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OFFICE	8503200251	850311			
SURNAME	PDR	ADOCK	05000320		
DATE	R		PDR		

TMI OCCUPATIONAL DOSE:

Licensee records indicate the following occupational exposures for January 1985.

Unit 2 (January estimate)	40.936 Rem
Station (January TLD)	51.042 Rem

January 1985 Station TLD Ranges (Rem)

No measurable exposure	1,447
Exposure less than 0.1	262
0.1 to 0.25	88
0.25 to 0.5	45
0.5 to 0.75	16
0.75 to 1	2
1 to 2	1
2 to 3	0

FUTURE MEETING:

On March 13, 1985 William Travers, Deputy Program Director, TMI Program Office, will speak to the Myerstown, Pennsylvania Rotary Club. Dr. Travers will discuss the NRC's role in the cleanup at Three Mile Island, Unit 2.

ORIGINAL SIGNED BY:
William D. Travers

William D. Travers
Deputy Program Director
TMI Program Office

Attachments:

1. Liquid Effluent and Environmental Data
2. Plant Status

OFFICE	TMIPD <i>DC</i>	TMIPD <i>CC</i>	TMIPD <i>Perant</i>	TMIPD <i>W.D. Travers</i>		
SURNAME	DCollins.ms.	CCowgill	Perant	W.D. Travers		
DATE	3/11/85	3/11/85	3/11/85	3/11/85		

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NRC PDR
LOCAL PDR
THI-2 Project
Section File

OFFICE ▶						
SURNAME ▶						
DATE ▶						

ATTACHMENT 1

LIQUID EFFLUENT AND ENVIRONMENTAL DATA

Environmental Protection Agency

Based on EPA's sampling results, liquid effluents being released from the TMI site are within regulatory limits and in accordance with NRC requirements and the City of Lancaster Agreement.

Lancaster Water Sample: Composite sample taken over seven days

Period Covered: February 17, 1985 - February 23, 1985

Results: Gamma Scan Negative for reactor related radioactivity

TMI Water Samples: Seven daily composited samples

Period Covered: February 16, 1985 - February 23, 1985

Results: Gamma Scan Negative for reactor related radioactivity

NRC Environmental Data

The NRC operated continuous outdoor air sampler at the TMI site did not detect any reactor related radioactivity.

<u>Sample</u>	<u>Period</u>
HP-460	February 27, - March 6, 1985

<u>Volume</u>	<u>Results</u>
342.5 m ³	LLD = 1.0 E-13 uCi/cc I-131 LLD = 1.0 E-13 uCi/cc Cs-137

ATTACHMENT 2

PLANT STATUS

Reactor Vessel Configuration: Reactor vessel open with modified internals indexing fixture installed

Core Cooling Mode: Heat transfer from the reactor coolant system (RCS) to reactor building ambient

Available Core Cooling/Makeup Sources:
Standby pressure control (SPC) system
Reactor coolant bleed tank (RCBT) water transfer system
Mini decay heat removal (MDHR) system
Decay heat removal (DH) system

Reactor Coolant System:

Average Cold Leg Temperature: 60°F

Reactor Core:

Average Incore Thermocouples:* 84°F
Maximum Incore Thermocouple:* 96°F

Reactor Building:

Temperature: 58°F
Pressure: -0.12 psig

Airborne Radionuclide Concentrations:

Tritium: 3.2 E-8 uCi/cc
Particulates: 1.5 E-11 uCi/cc predominately Cs-137

*Uncertainties exist as to the exact location and accuracy of these readings.