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January 28, 1985  
NRC/TMI-85-009

MEMORANDUM FOR: Harold R. Denton, Director  
Office of Nuclear Reactor Regulation  
Bernard J. Snyder, Program Director  
TMI Program Office

FROM: William D. Travers, Deputy Program Director  
TMI Program Office

SUBJECT: NRC TMI PROGRAM OFFICE WEEKLY STATUS REPORT FOR  
JANUARY 20, 1985 - JANUARY 26, 1985

Plant parameters have shown no significant changes. Waste processing continued on a routine basis. Other site activities this period included: continued fuel pool "A" refurbishment, and building decontamination.

Significant items covered in the enclosure are:

- Reactor Building Activities
- Auxiliary and Fuel Handling Building Activities
- Plant Status Data
- Public Meeting

The summary sheet included in this report is:

- Liquid Effluent and Environmental Data

William D. Travers  
Deputy Program Director  
TMI Program Office

Enclosure: As stated

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## ENCLOSURE

### REACTOR BUILDING ACTIVITIES:

During this week the fuel transfer upender mechanism was removed from the reactor building canal in preparation for installation of new fuel canister transfer equipment. Dose reduction activities continue with scabbling of floor surfaces on the 305' elevation. Pre-requisites for the scheduled May 1985 plenum removal also include installation of the canal dam and flooding of the deep end of the canal.

### AUXILIARY AND FUEL HANDLING BUILDING ACTIVITIES:

Refurbishment of the "A" fuel pool continues with re-installation of the fuel transfer mechanism rails. Operation of the purification demineralizer cesium elution system has resumed. Replacement of the construction cork seal between the Reactor Building and the Auxiliary Building has continued. Decontamination flushing in the make-up valve alley has continued.

### PLANT STATUS:

The reactor vessel head is removed and the RCS is open to ambient with the modified internals indexing fixture installed. The reactor core decay heat is being transferred by the reactor coolant system to the building ambient. Core cooling/makeup is available from the standby pressure control system, the reactor coolant bleed tank water system and the mini decay heat removal system.

The average reactor coolant cold leg temperature is 59°F. The average of the reactor core thermocouples is 83°F, with a maximum thermocouple of 91°F (uncertainties exist as to their exact location and accuracy). The reactor building temperature is about 58°F and pressure is about -.05 psig. Of reactor building samples this week, airborne tritium was 1.7 E-7 uCi/cc and particulates, predominately cesium-137, were 8.1 E-11 uCi/cc.

### PUBLIC MEETINGS:

On February 14, 1985, the Advisory Panel for the Decontamination of Three Mile Island Unit 2 will meet from 7:00 PM to 10:00 PM in the Holiday Inn, 23 South Second Street, Harrisburg, Pennsylvania. The meeting will be open to the public.

Persons desiring the opportunity to speak before the Panel are asked to contact Mr. Thomas Smithgall at 717-291-1042 or write to him at 2122 Marietta Avenue, Lancaster, Pennsylvania 17603. Persons desiring to submit topics or questions for consideration by the Panel are asked to contact, in writing, Mayor Arthur Morris, 120 North Duke Street, Lancaster, Pennsylvania 17602.



APPENDIX

LIQUID EFFLUENT AND ENVIRONMENTAL DATA

Environmental Protection Agency

Lancaster Water Sample: Composite sample taken over seven days  
Period Covered: January 6, 1985 - January 12, 1985  
Results: Gamma Scan Negative for reactor related radioactivity

TMI Water Samples: Seven daily composited samples  
Period Covered: January 5, 1985 - January 12, 1985  
Results: Gamma Scan Negative for reactor related radioactivity

NRC Environmental Data

The NRC operated continuous outdoor air sampler at the TMI site did not detect any reactor related radioactivity.

| <u>Sample</u>        | <u>Period</u>         | <u>Results</u>        |        |
|----------------------|-----------------------|-----------------------|--------|
| HP-454               | January 16 - 23, 1985 | LLD = 1.1 E-13 uCi/cc | I-131  |
|                      |                       | LLD = 1.1 E-13 uCi/cc | Cs-137 |
| <u>Volume</u>        |                       |                       |        |
| 315.9 m <sup>3</sup> |                       |                       |        |